

# **Metallic Fuel Development**

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**Argonne National Laboratory**

**AFCI Semi-Annual Review Meeting**  
**Santa Fe, NM**  
**August 27, 2003**

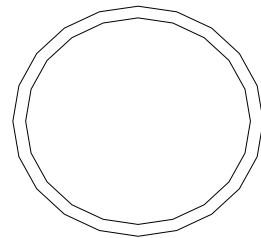


# Outline of Presentation

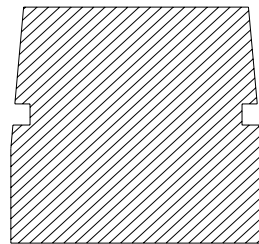
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- **Fabrication**
  - **Metallic Fuel Fabrication Process Development**
  - **Test Fuels and Sample Production**
  - **Rodlet Fabrication (Metallic and Nitride Fuels)**
- **Characterization**
  - **Chemical Analysis & Isotopics on Fuels**
  - **Microstructural Analysis**
  - **Thermal Analysis**
  - **Fuel-Cladding Compatibility**
- **Irradiation Test Fabrication and Assembly**
- **Postirradiation Examination**

# Metallic Fuel Fabricated by Arc-Casting

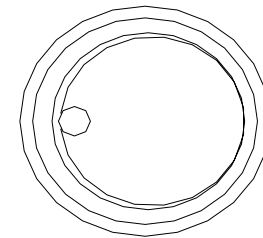


Top View

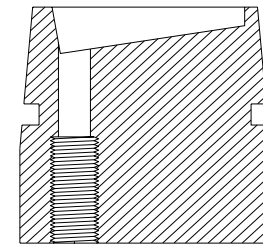


Section View

Blank  
Hearth



Top View



Section View

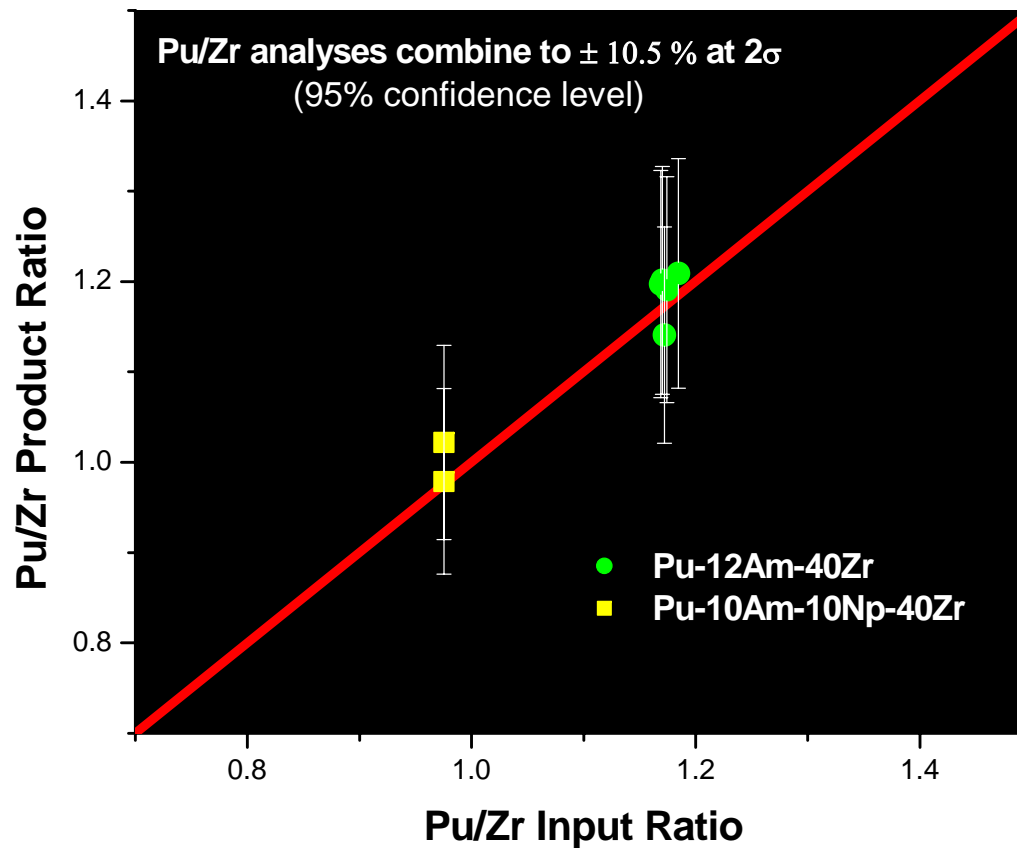
Quartz Tube

Tube Holder

End Plug

Modified Hearth

# No Americium Loss During Arc-Casting

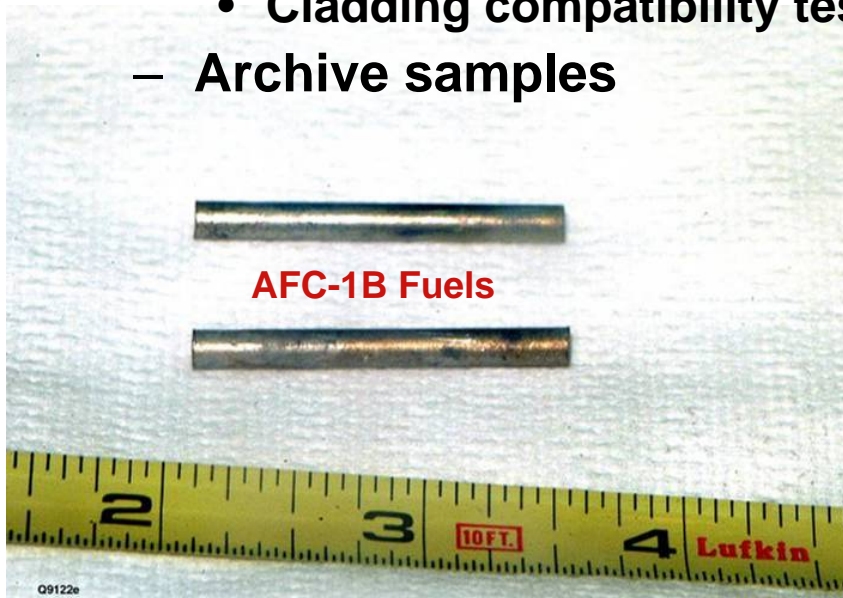


# Metallic Fuel Fabrication



# Metallic Fuels for AFC-1 Tests

- **Arc-Cast Pins Sectioned to Provide**
  - Fuel specimens for irradiation tests
  - Samples for chemical and isotopic analysis
  - Samples for characterization
    - Microstructural analysis (XRD/SEM)
    - Thermal Analysis (DSC/DTA/TGA)
    - Cladding compatibility testing (diffusion couples)
  - Archive samples



# Metallic Fuel Rodlet Fabrication

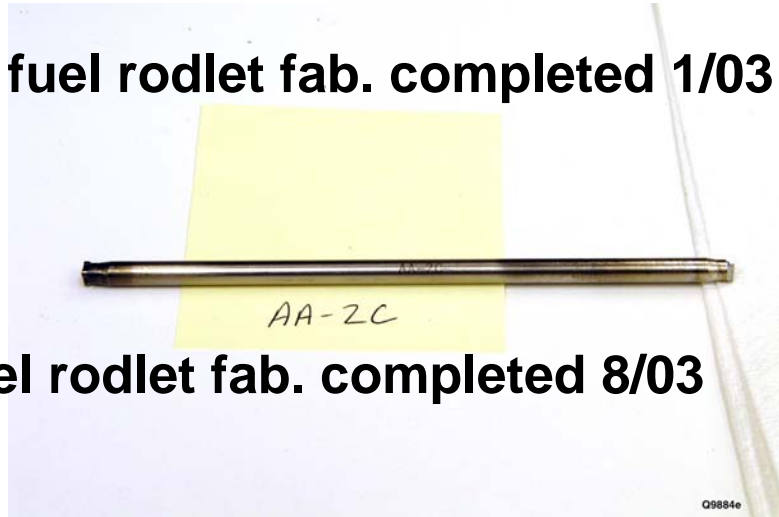
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1. Fabricate jackets; load and settle sodium.
2. Load fuel on top of solid sodium; weld top end plug.
3. Weld inspection: visual exam, He leak testing, radiography.
4. Sodium heated above melting temperature; fuel column settled to bottom end plug.
5. Radiographic inspection of sodium level and slug position.
6. Fuel rodlet heated to  $\sim 500^{\circ}\text{C}$ ; sodium wets fuel and cladding.
7. Radiographic inspection of final slug position and sodium height.

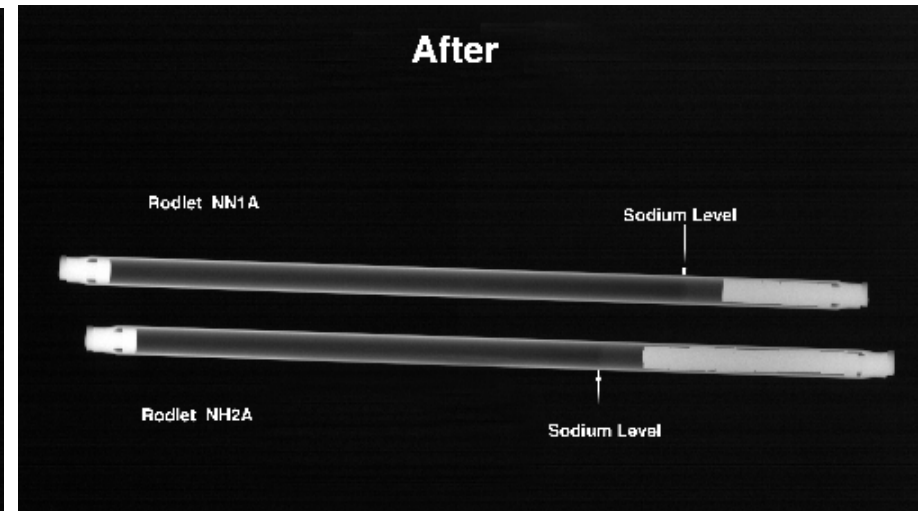
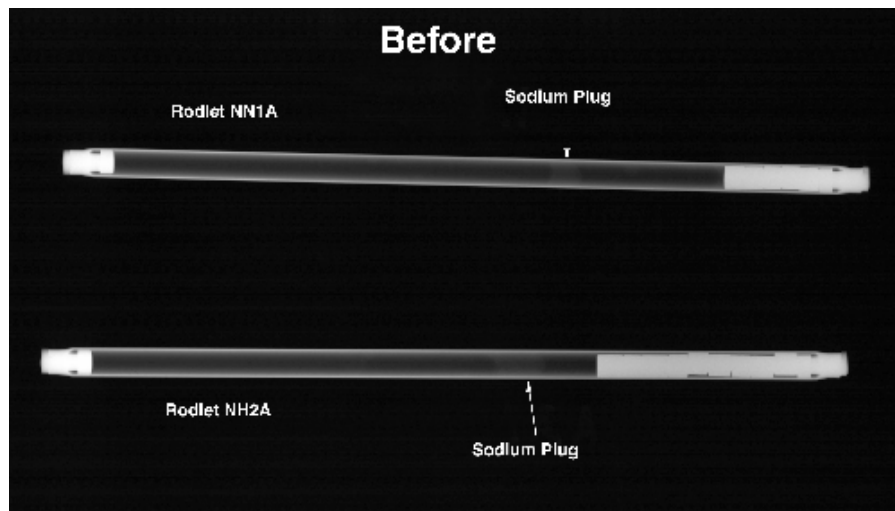
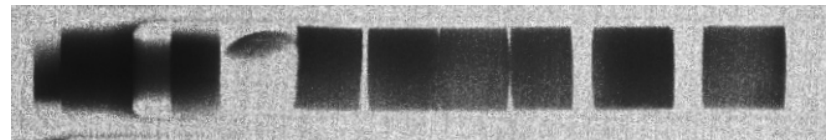
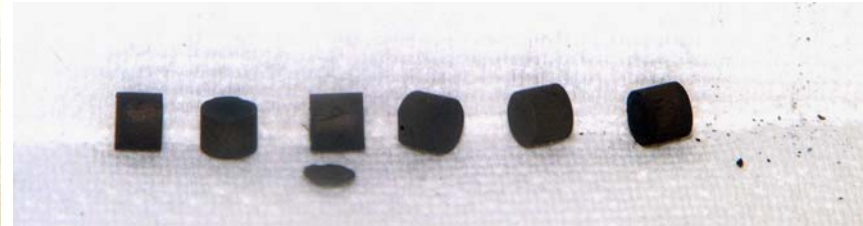
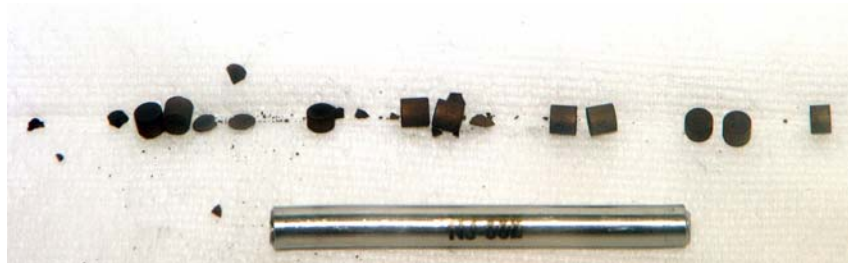
# Fuel Rodlet Fabrication Status

- AFC-1B,D non-fertile **metallic** fuel rodlet fab. completed 1/03
  - Pu-12Am-40Zr (2)
  - Pu-10Am-10Np-40Zr
  - Pu-40Zr
  - Pu-60Zr
- AFC-1F low-fertile **metallic** fuel rodlet fab. completed 8/03
  - U-29Pu-4Am-2Np-30Zr (2)
  - U-25Pu-3Am-2Np-40Zr (2)
  - U-34Pu-4Am-2Np-20Zr
  - U-28Pu-7Am-30Zr
- AFC-1A,C non-fertile **nitride** fuel rodlets fabricated 4/03
  - Rodlets not usable due to pellet fragmentation
  - AFC-1C (high BU) eliminated
  - AFC-1A consolidated with AFC-1E to AFC-1Æ
- AFC-1Æ non/low-fertile **nitride** fuel rodlet fabrication in progress but will likely not meet 09/03 milestone





# Nitride Rodlet Fabrication Demonstrated

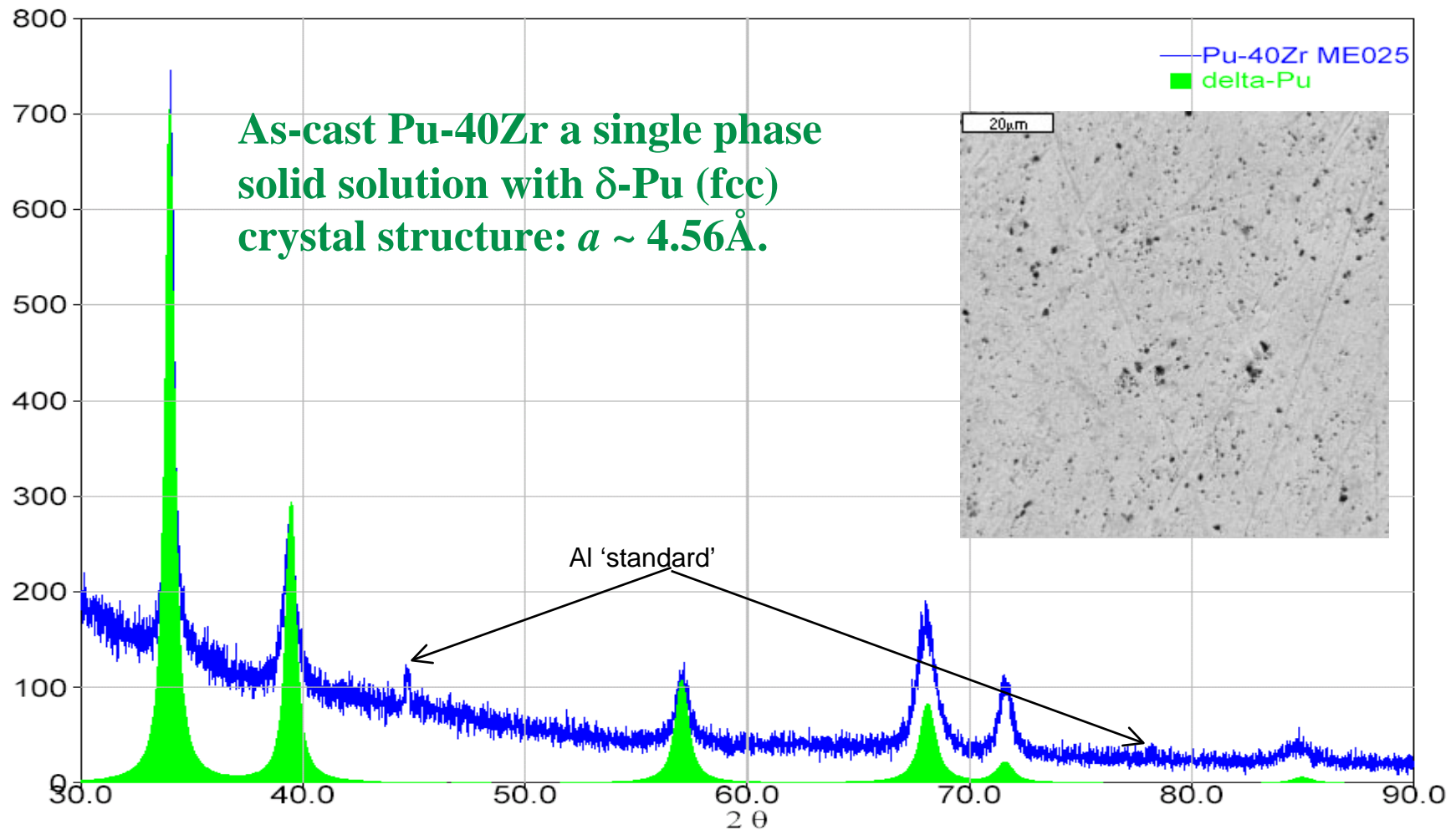


# Metallic Fuel Characterization

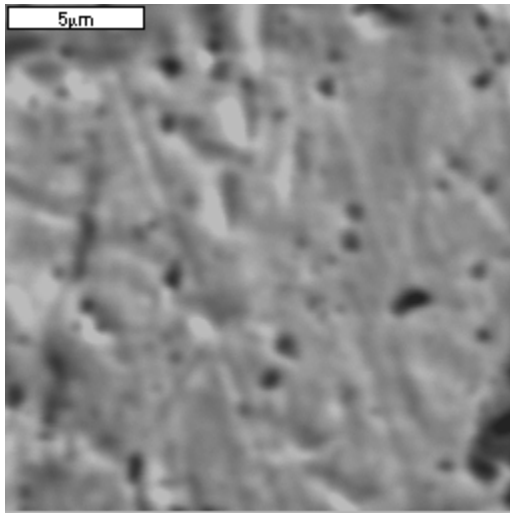
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- **Non-Fertile Metallic Alloys (FY03 complete)**
  - Pu-12Am-40Zr
  - Pu-10Np-40Zr
  - Pu-10Am-10Np-40Zr
  - Pu-40Zr
  - Pu-60Zr
- **Low-Fertile Metallic Alloys (in progress)**
  - U-29Pu-4Am-2Np-30Zr
  - U-25Pu-3Am-2Np-40Zr
  - U-34Pu-4Am-2Np-20Zr
  - U-28Pu-7Am-30Zr

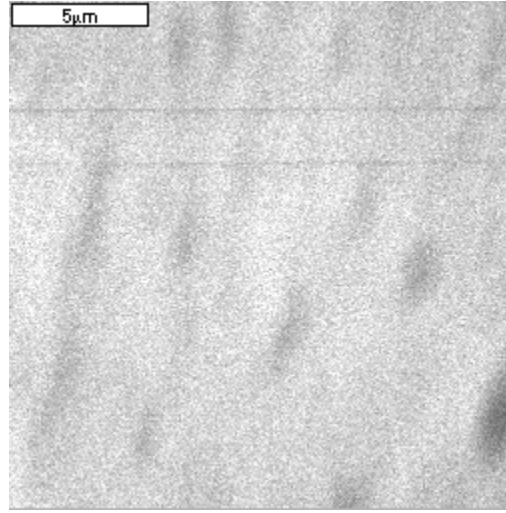
# Microstructural Analysis of Pu-40Zr



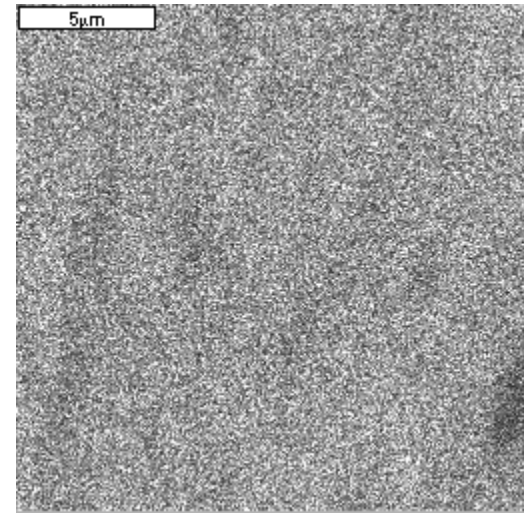
# X-Ray Maps of Pu-40Zr



BSE Image



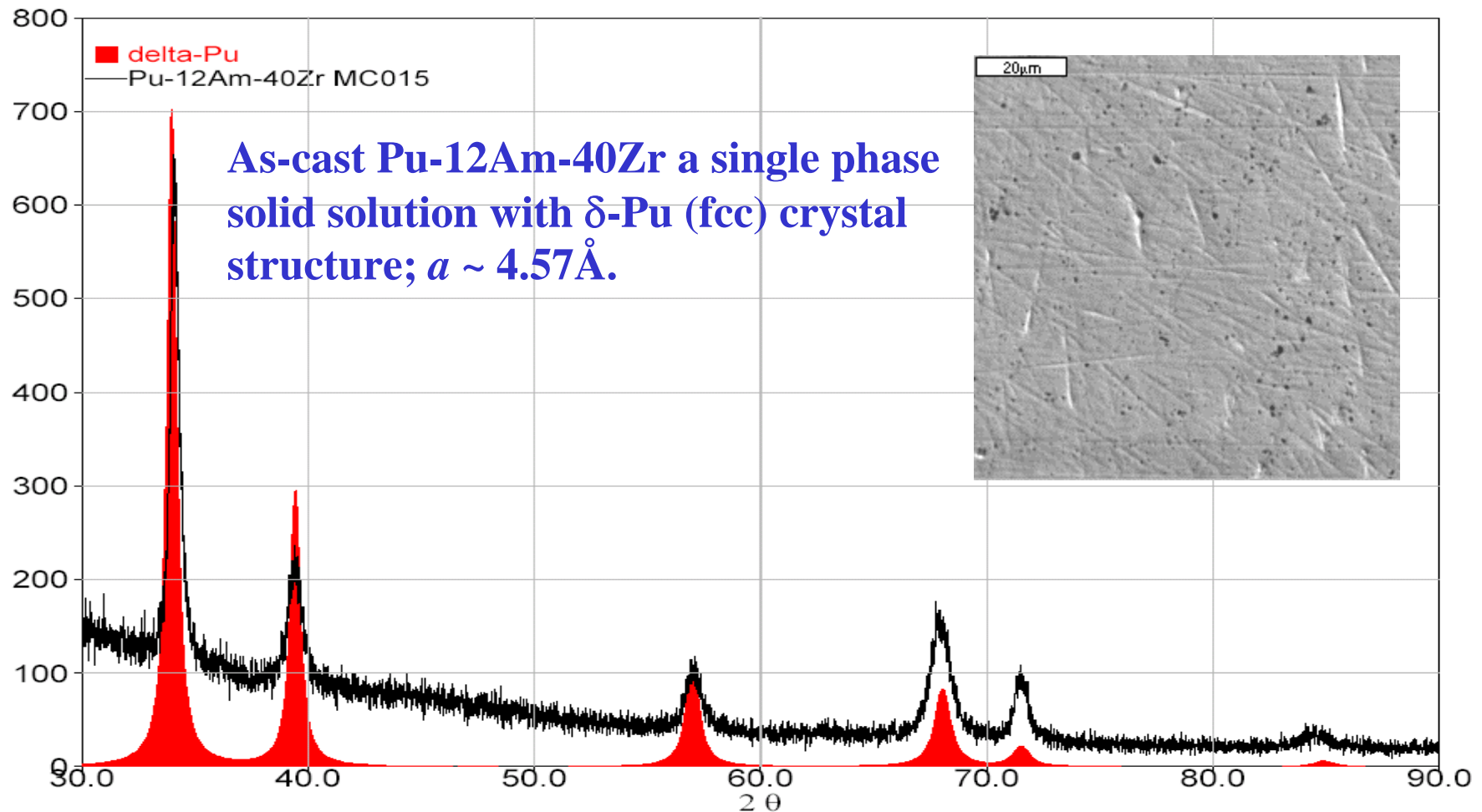
Pu



Zr

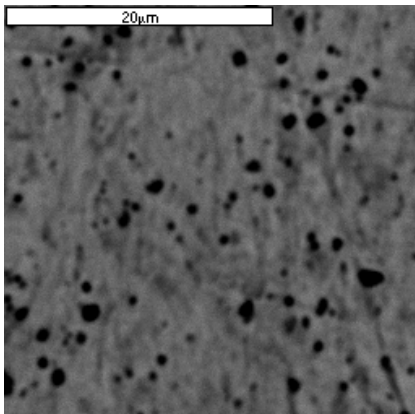
- Alloy is single-phase
- Even distribution of Pu and Zr
- Agrees with published Pu-Zr phase diagram

# Microstructural Analysis of Pu-12Am-40Zr

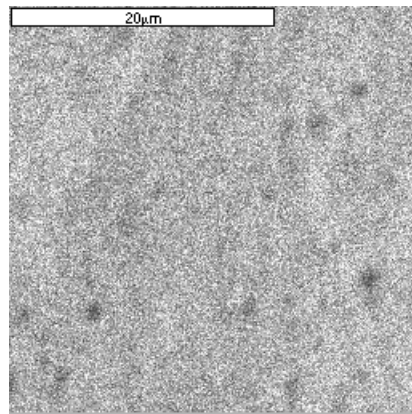




# X-Ray Maps of Pu-12Am-40Zr

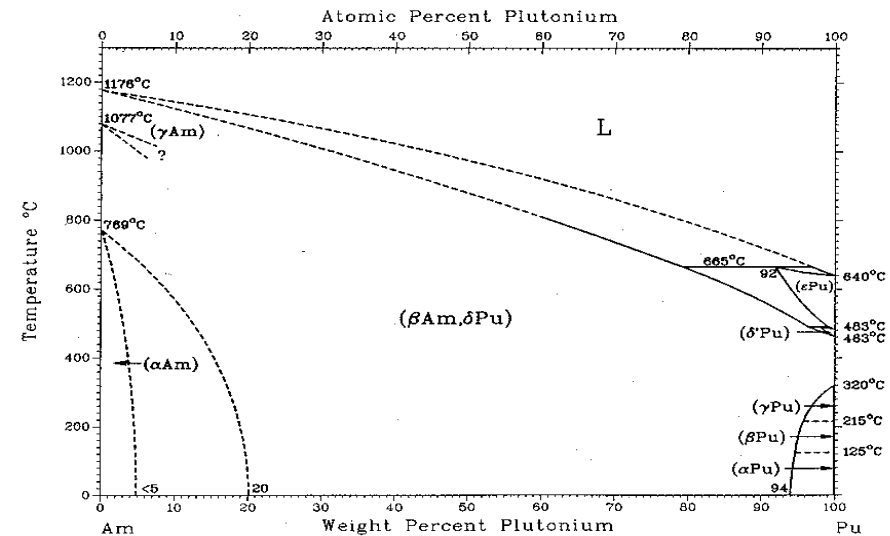


BSE Image

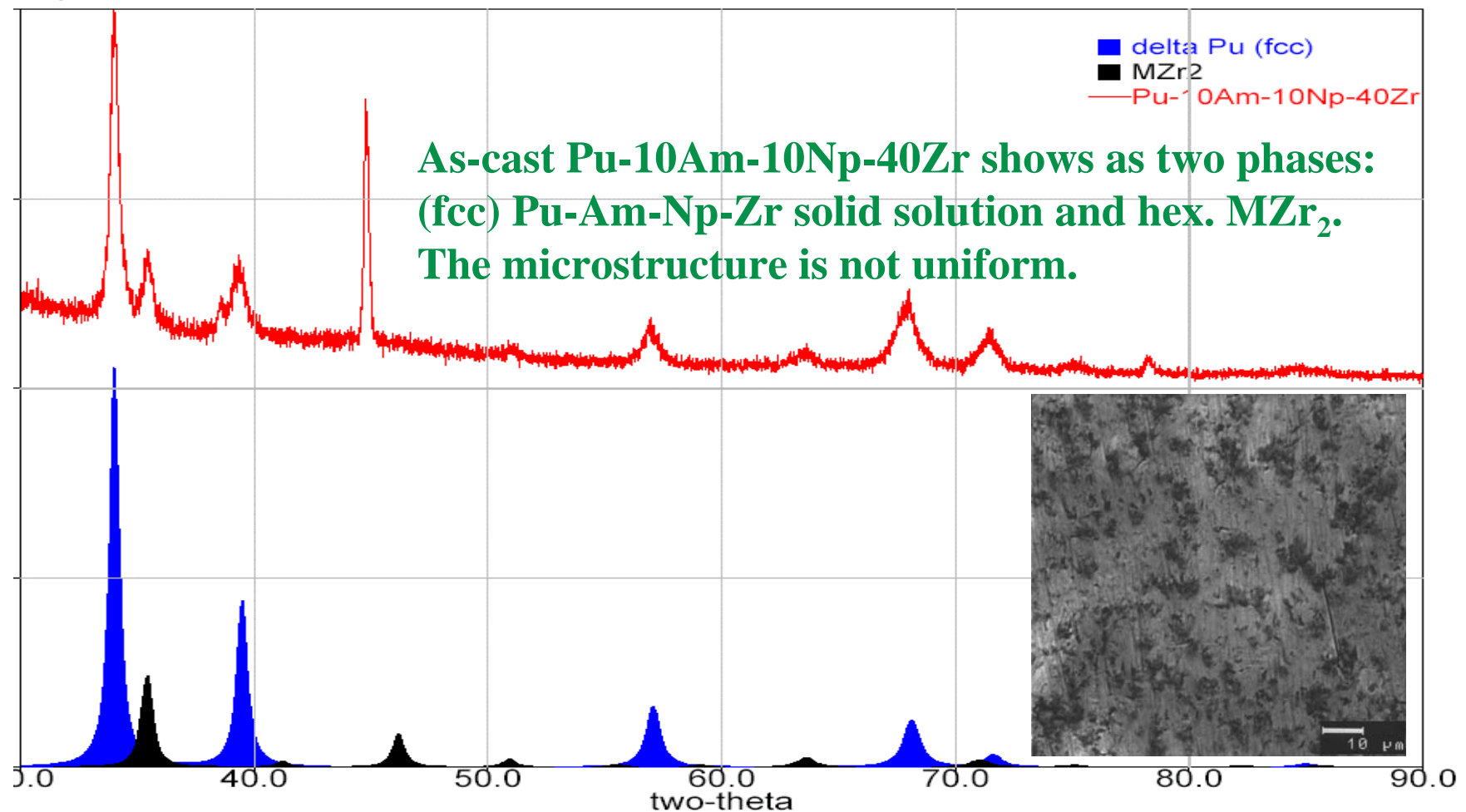


Pu

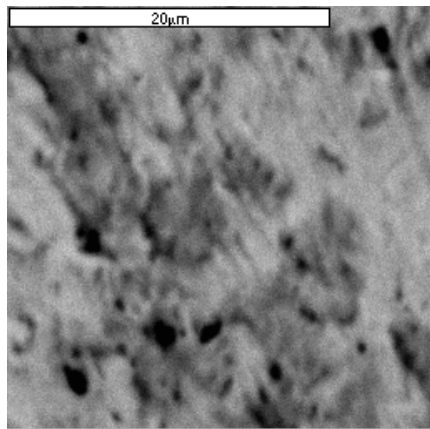
- Alloy is single-phase
- Even distribution of Pu, Am, and Zr
- Am stabilizes  $\delta$ -Pu



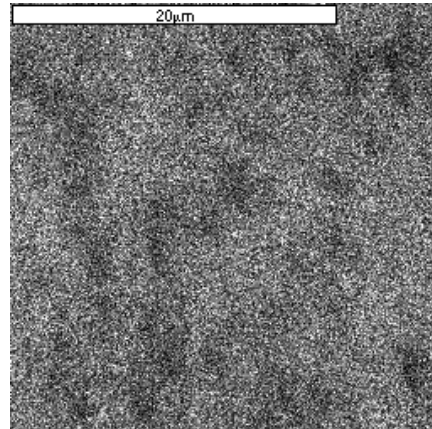
# Microstructural Analysis of Pu-10Am-10Np-40Zr



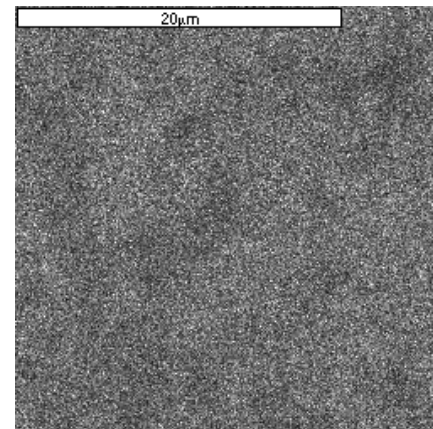
# X-Ray Maps of Pu-10Am-10Np-40Zr



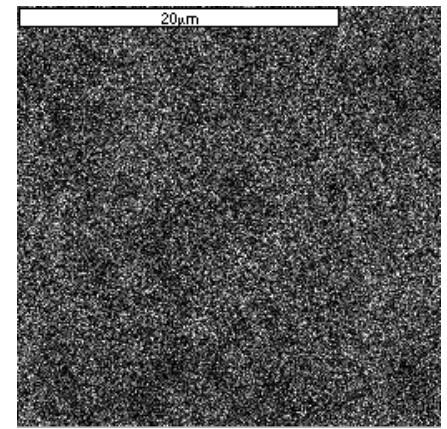
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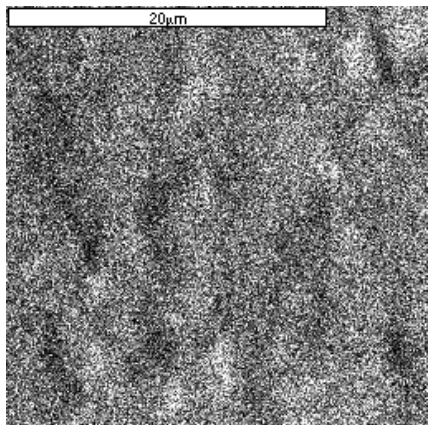
Pu



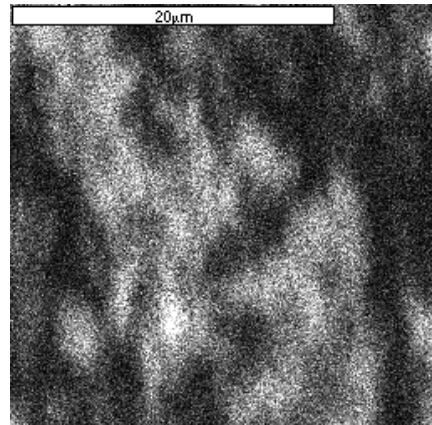
Np



Am



Zr



O

- Pu, Np, and Am found in same areas of microstructure.
- Zr depleted in areas enriched in actinides.
- Globular phases enriched in oxygen, may be stabilizing the  $\text{MZr}_2$  phase.



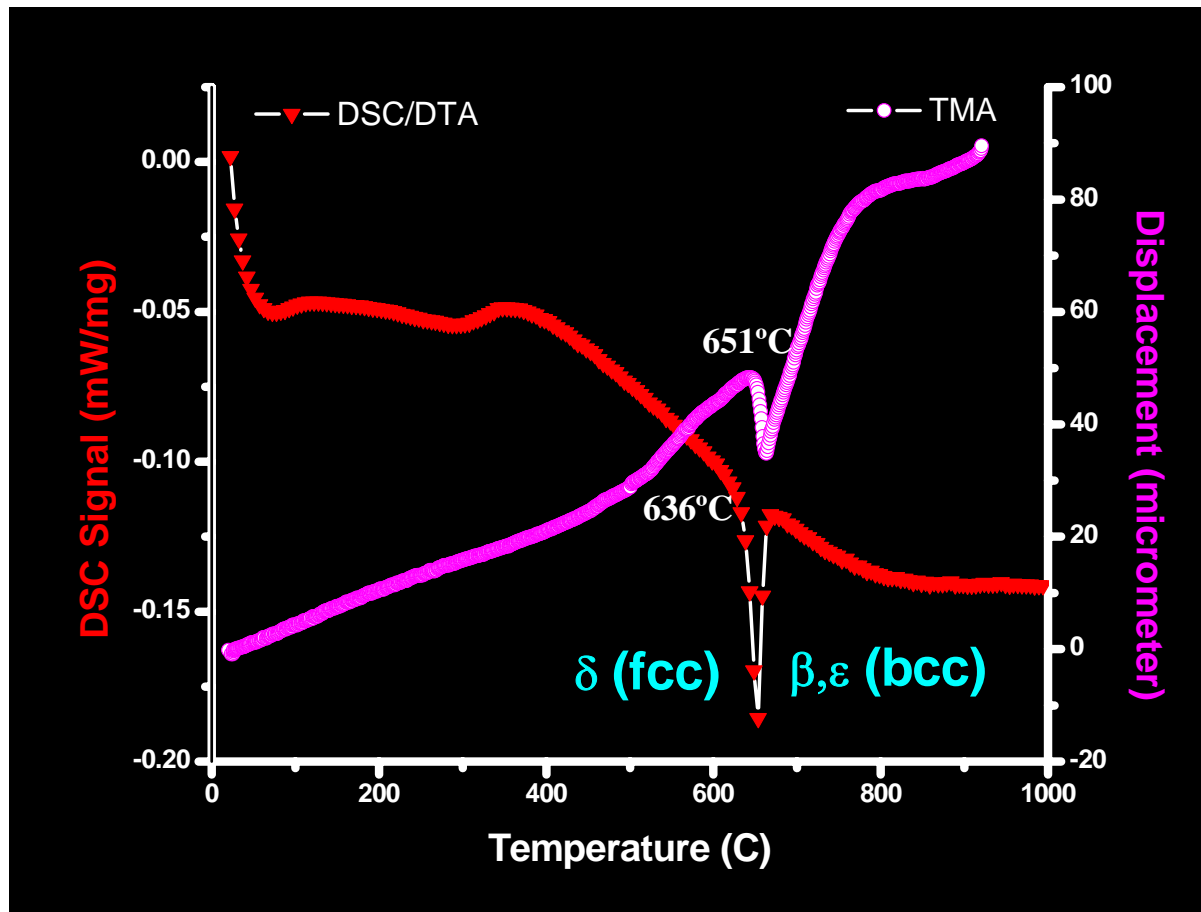
# Thermal Analysis of Non-fertile Alloys

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- **Identification of Phase Transitions up to 1000°C**
  - Pu-12Am-40Zr
  - Pu-10Np-40Zr
  - Pu-10Am-10Np-40Zr
  - Pu-40Zr
  - Pu-60Zr
  - No melting observed in these alloys to 1000°C
- **Analysis Techniques**
  - Differential Scanning Calorimetry (DSC)
  - Differential Thermal Analysis (DTA)
  - Thermo-Mechanical Analyzer (TMA)

# DSC/DTA/TMA Heating Curves

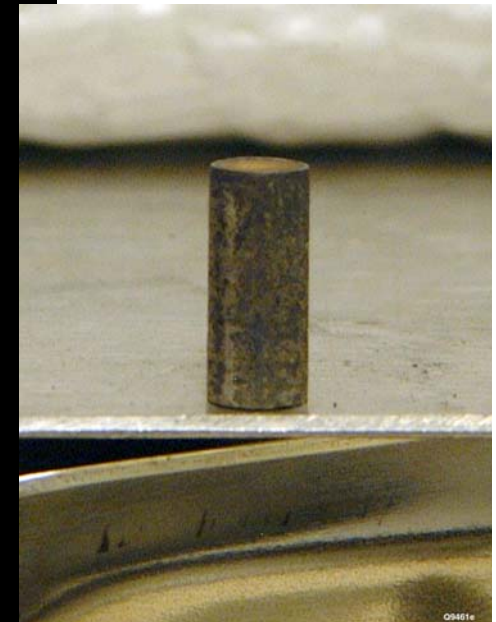
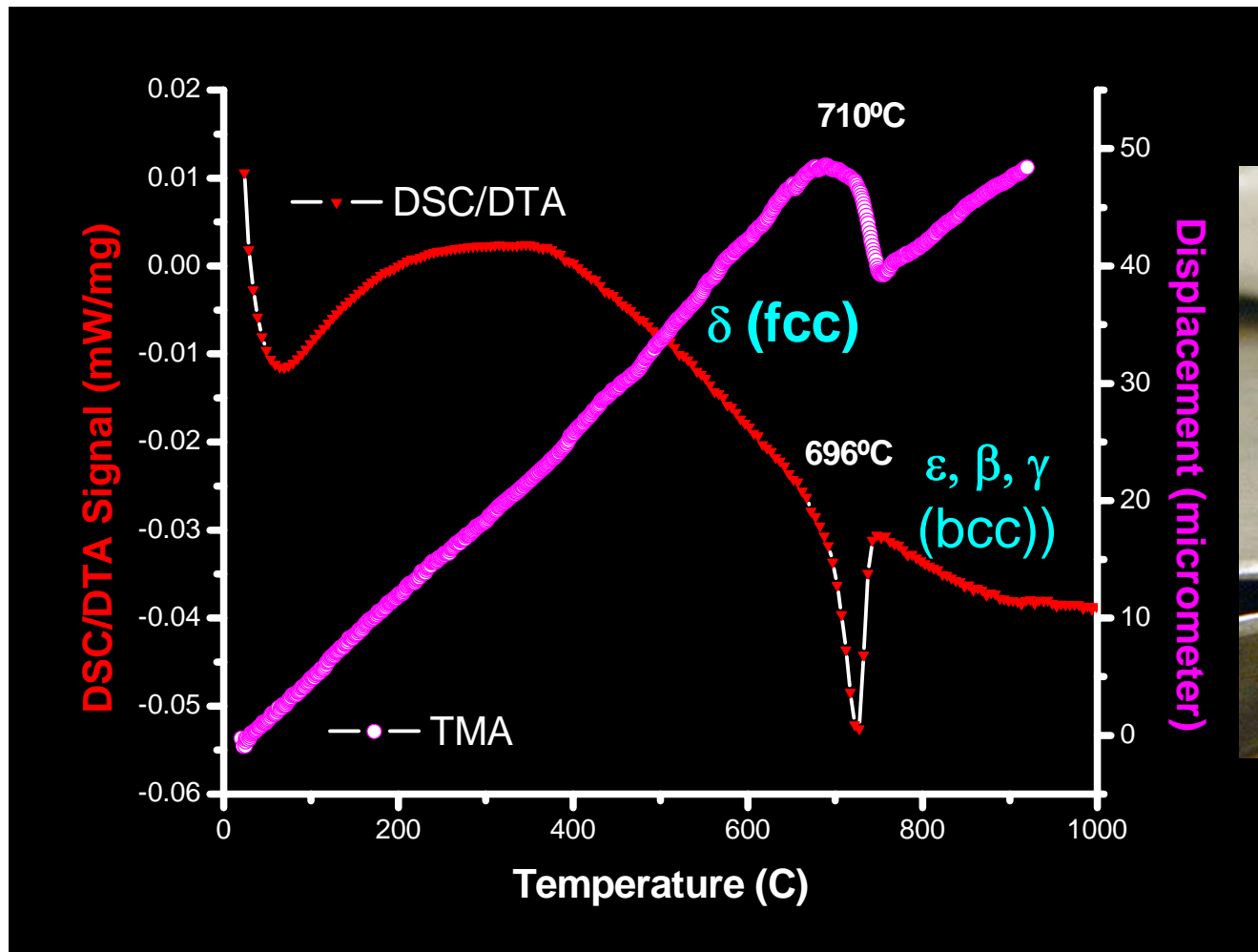
## Pu-40Zr



fcc to bcc transition from published phase diagram: **640°C**

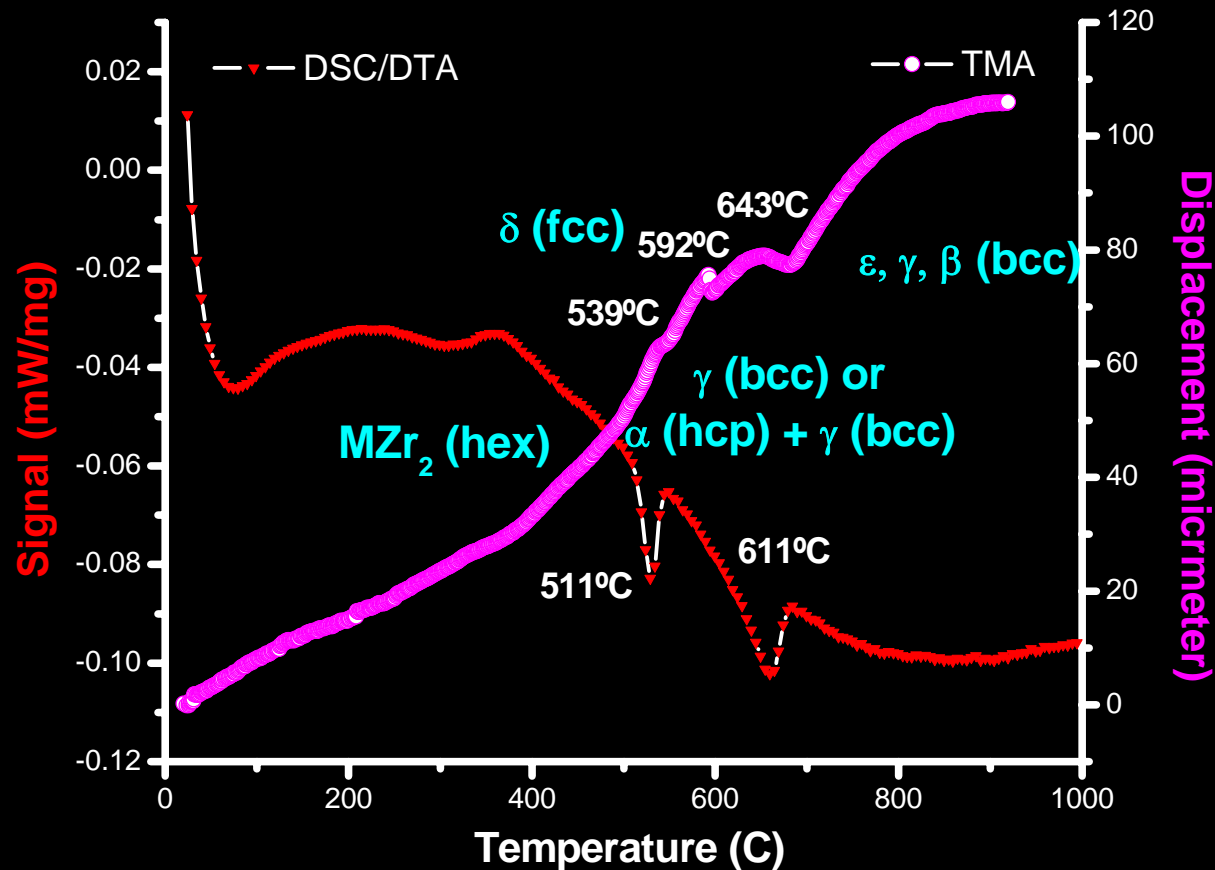
# DSC/DTA/TMA Heating Curves

## Pu-12Am-40Zr



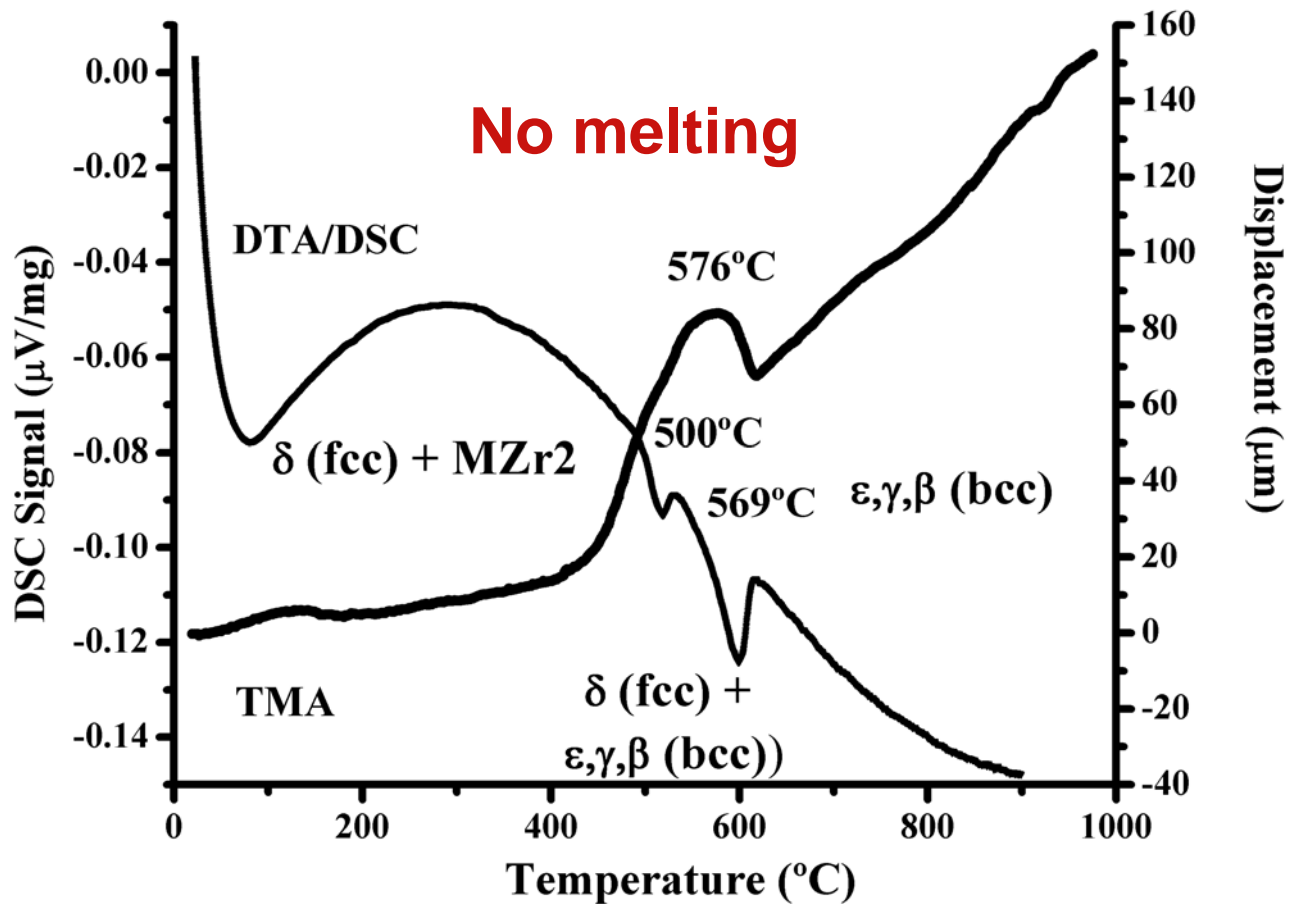
# DSC/DTA/TMA Heating Curves

## Pu-10Am-10Np-40Zr



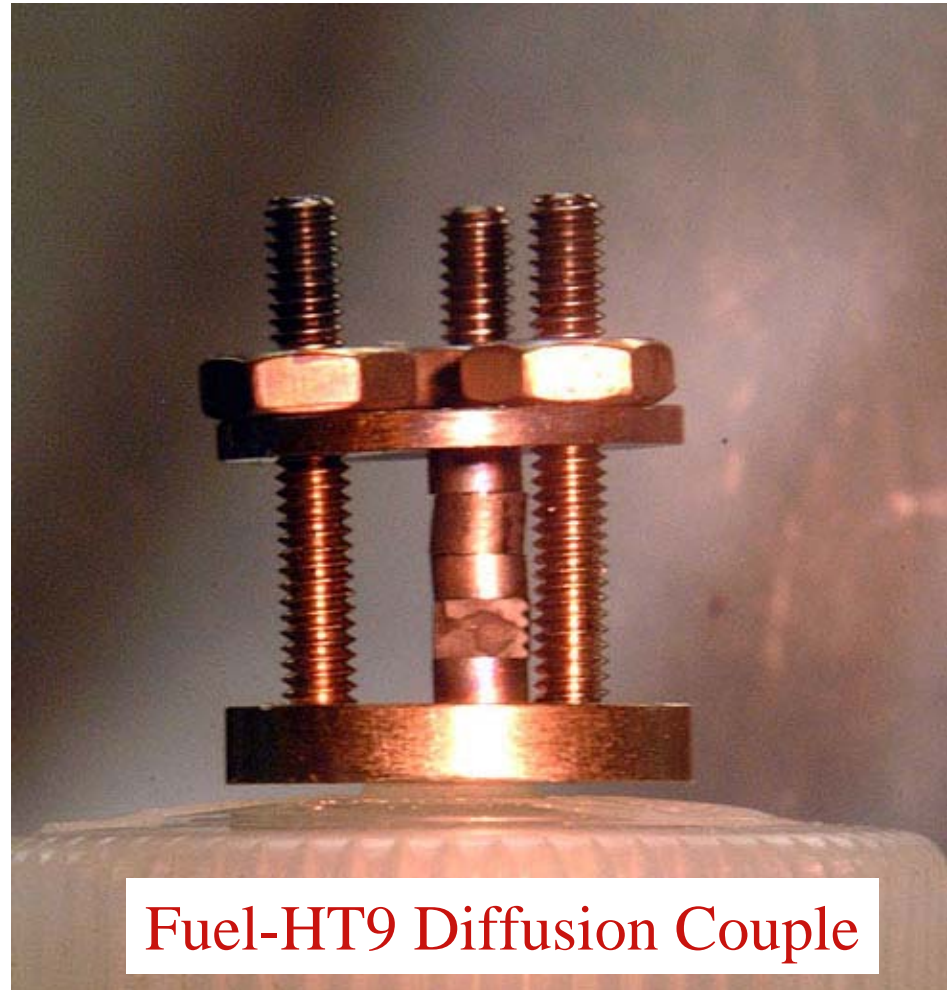
# DSC/DTA/TMA Heating Curves

## Pu-10Np-40Zr



# Fuel-Cladding Chemical Interaction

- **Diffusion Couples**
  - HT-9 (Type 422 SS) vs.
    - Pu-12Am-40Zr
    - Pu-10Np-40Zr
    - Pu-10Am-10Np-40Zr
    - Pu-40Zr
    - Pu-60Zr
  - Furnace anneals
    - 50 to 200 hours
    - 650°C
    - 850°C
  - Analysis
    - SEM/EDS/WDS



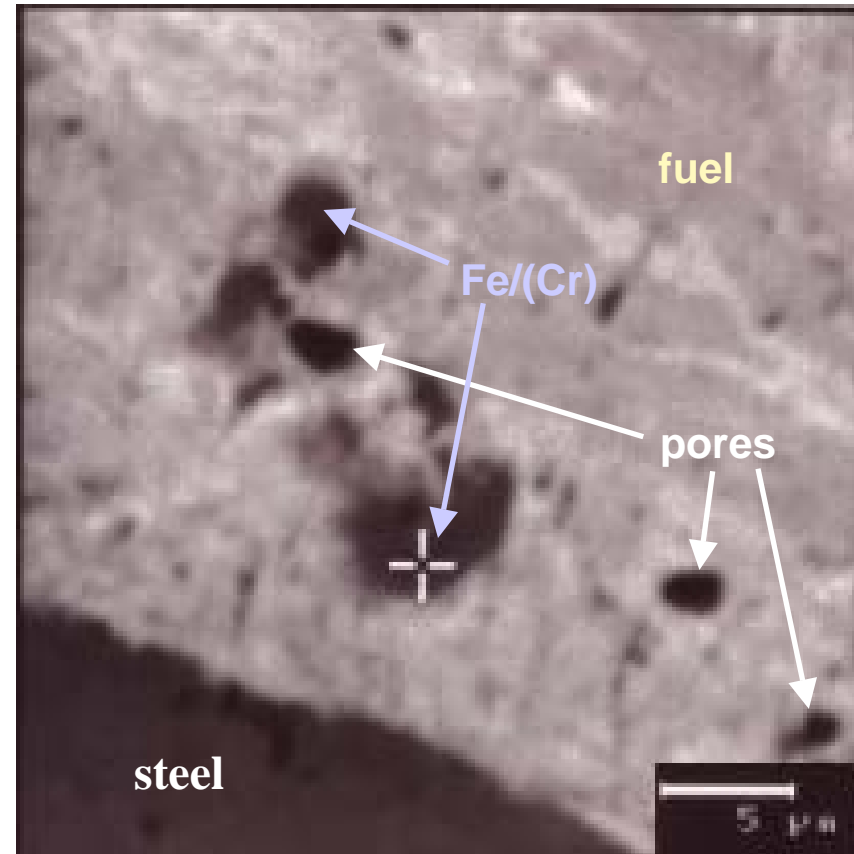
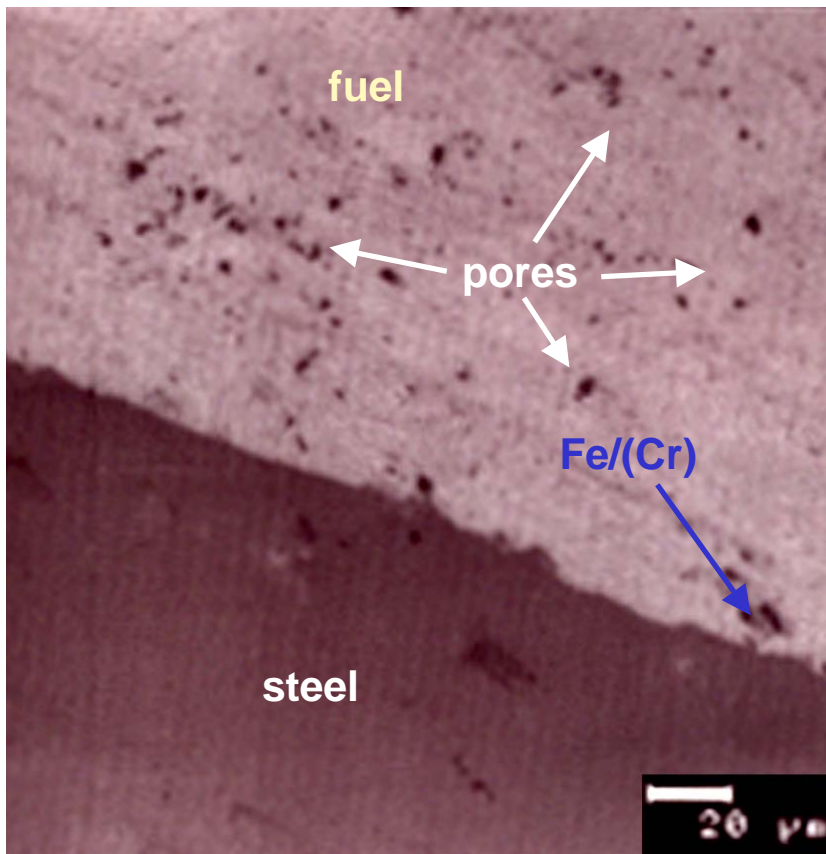
# FCCI Results

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- **At 650°C**
  - No uniform diffusion layers observed
  - Discrete Fe-rich globular precipitates observed penetrating into fuel alloy
- **At 850°C**
  - Complete interdiffusion of fuel alloy and cladding
  - Same discrete Fe-rich precipitates throughout fuel
  - Also discrete Zr-rich precipitates throughout fuel
  - No melting
- **Conclusions**
  - Fe-rich and Zr-rich phases have high melting points
  - No actinide-rich, low-melting phases observed
  - With no diffusion layers formed at fuel-clad interface, cladding should retain nominal mechanical properties



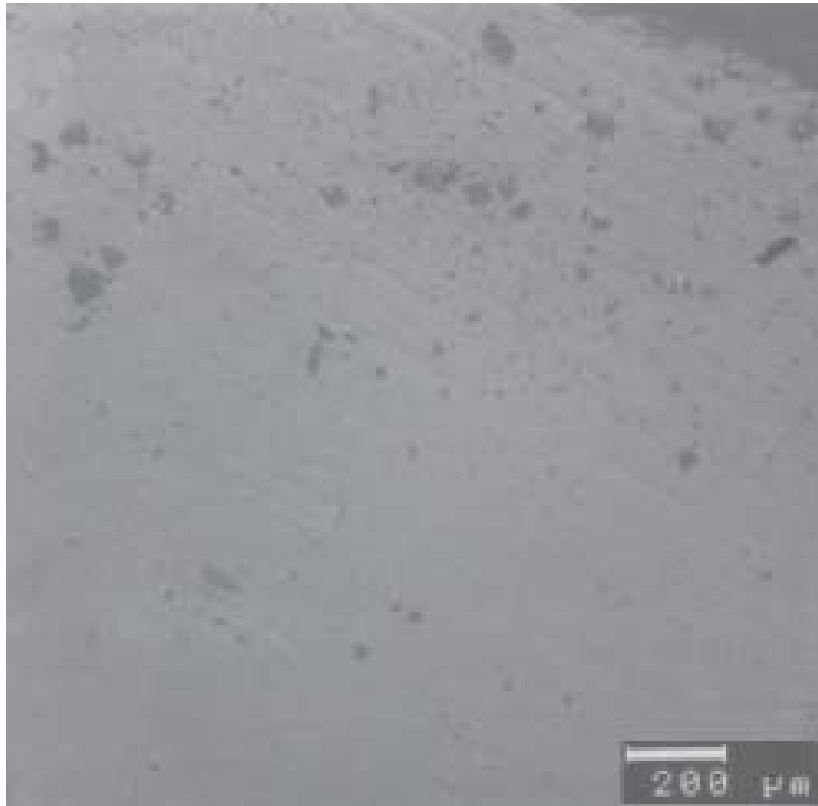
# BSE Micrographs of Diffusion Couples (650°C for 50 hours)



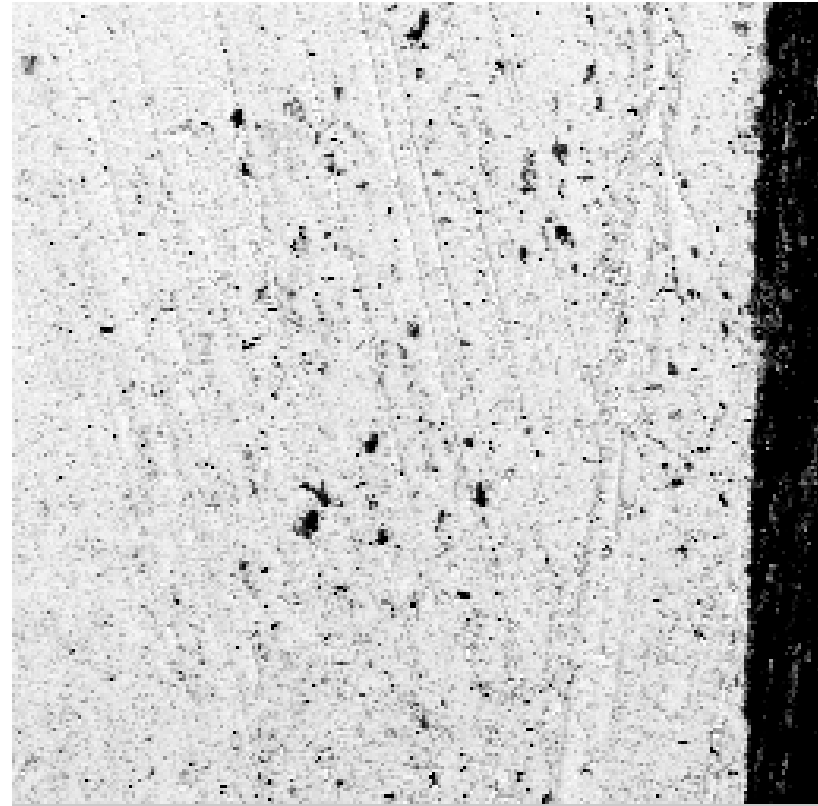
Pu-60Zr/422



# BSE Micrographs of Diffusion Couples (650°C for 200 hours)



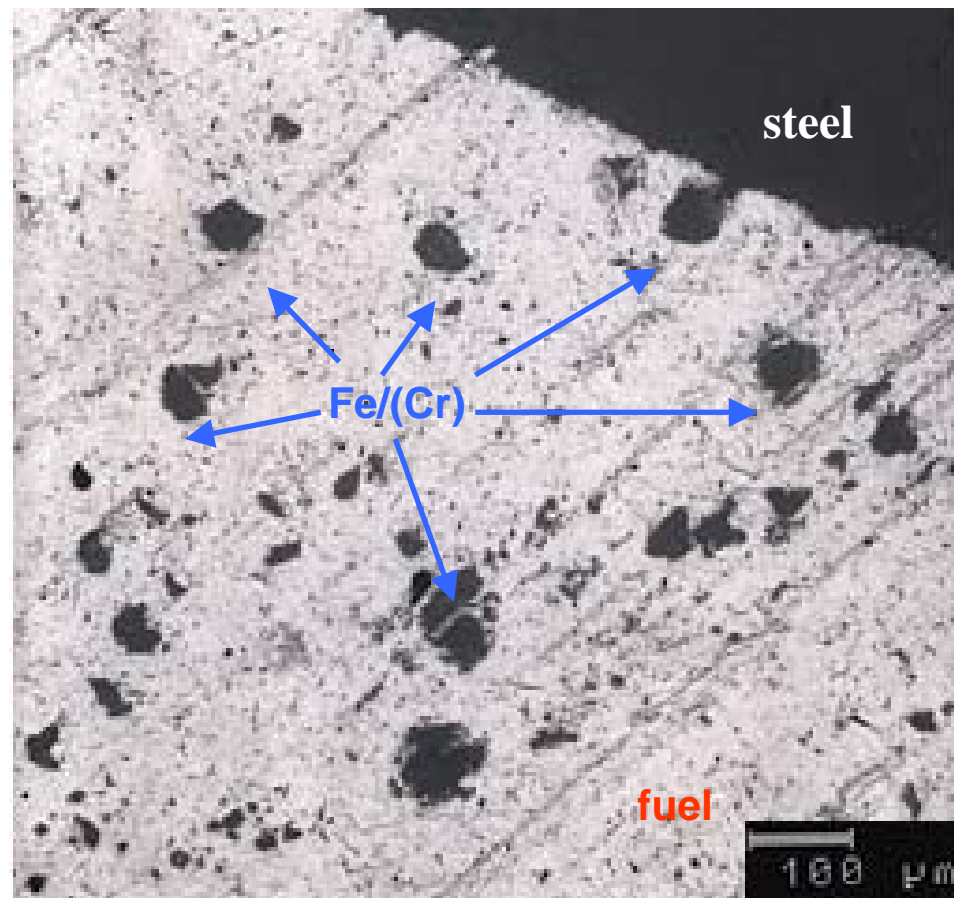
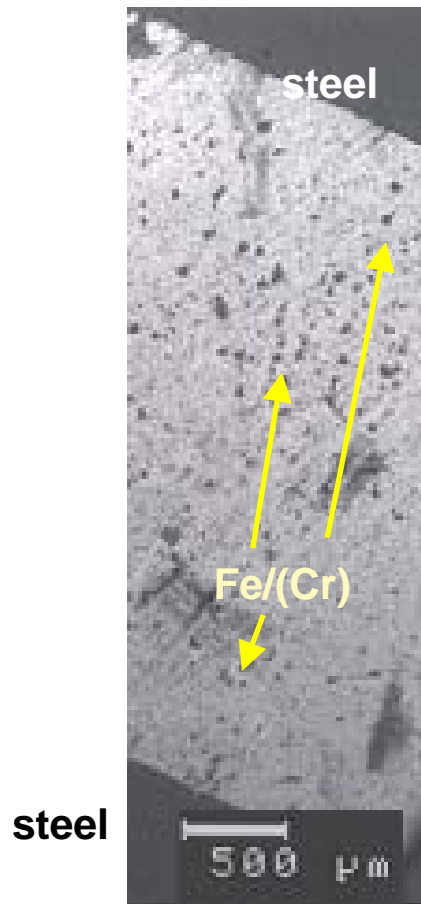
Pu-40Zr/422



Pu-10Am-10Np-40Zr/422

- Fe/(Cr) deposits observed out to 1000 microns of interface
- Other dark features throughout fuel are pores

# BSE Micrographs of Diffusion Couples (860°C for 50 hours)

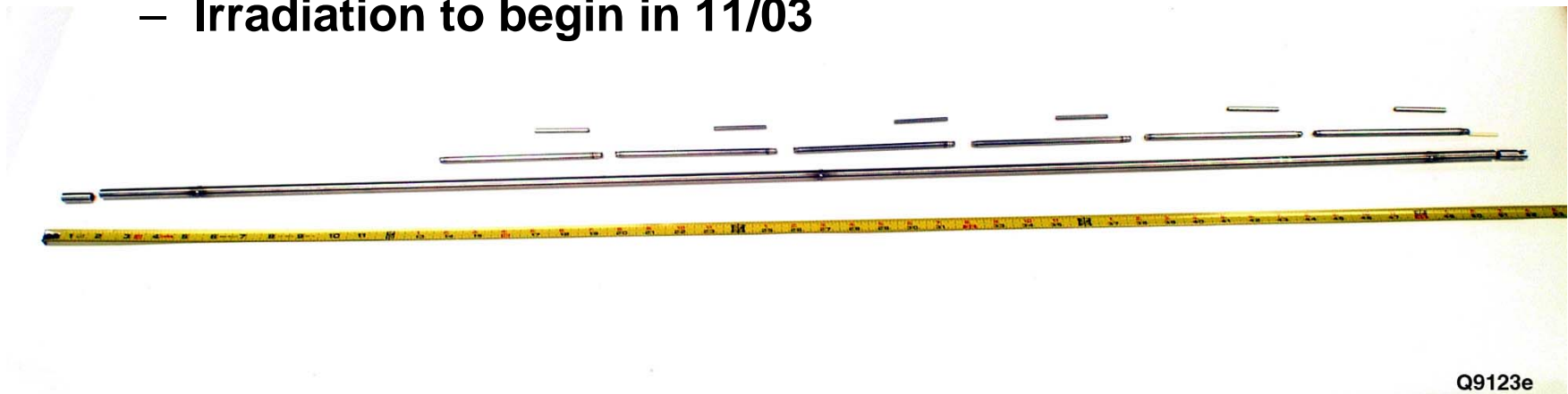


**Pu-40Zr/422**

Complete interdiffusion in couples with Pu-10Np-40Zr, Pu-12Am-40Zr and Pu-10Am-10Np-40Zr with similar structures

# Irradiation Test Fabrication/Assembly

- **AFC-1B, -1D Test Capsules**
  - Fabricated and assembled 5/03
  - Delivered to ATR on June 3, 2003
  - Irradiation in progress
- **AFC-1Æ, -1F Test Capsules**
  - Hardware fabrication complete
  - Assembly scheduled for 9/03
  - Irradiation to begin in 11/03



# Postirradiation Examination

- **Plans for Postirradiation Examination**
  - AFC-1A,B,C,D PIE Plan issued in January 2003
  - AFC-1E,F PIE Plan issued in May 2003
- **Preparations Initiated to Receive GE-2000 Cask at the Hot Fuel Examination Facility (June 2004)**

